

Title

Tokamak GOLEM

Vojtěch Svoboda

15. ledna 2012

Outline of the talk

1 Introduction

2 Tokamak GOLEM

3 Other Tokamaks

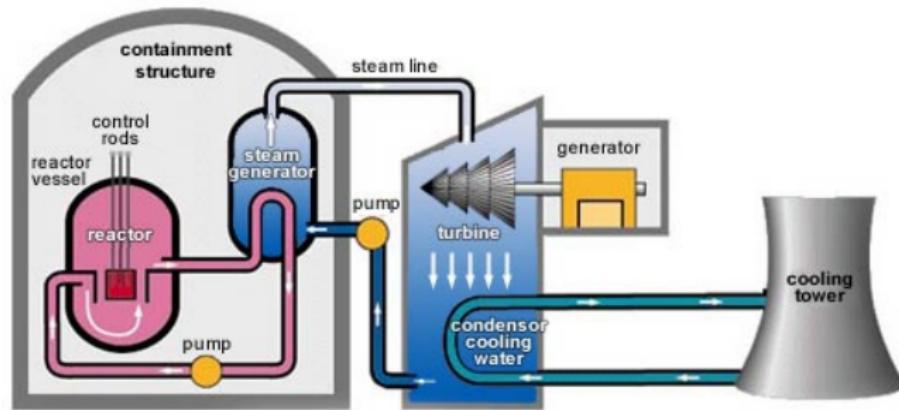
Content

1 Introduction

2 Tokamak GOLEM

3 Other Tokamaks

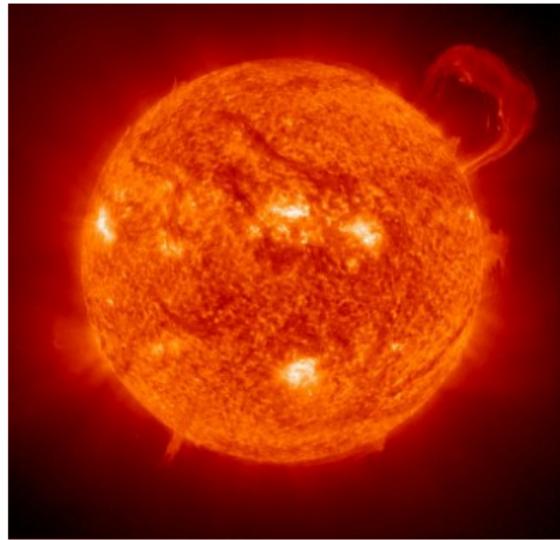
Thermal power plant - basic principle



The question:

?? WHAT TO BURN ??

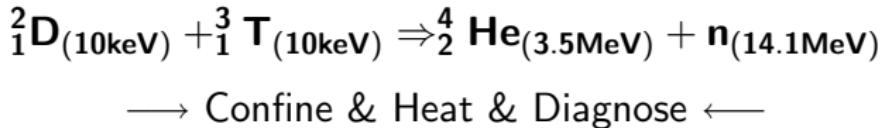
Harnessing the Sun's (star's) energy



Core Burning Stages in a 25 Solar Mass Star:

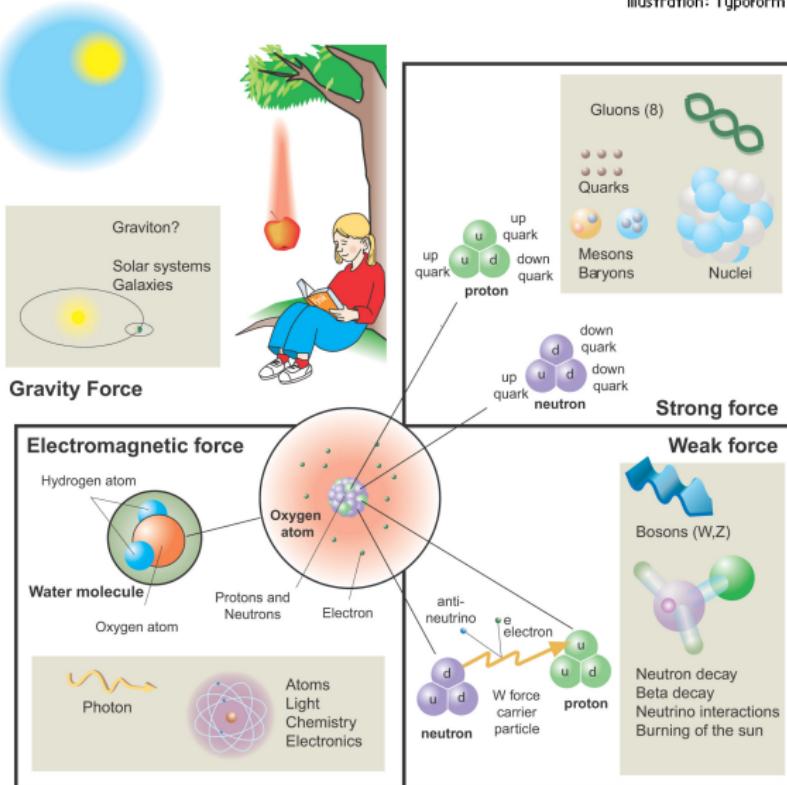
Fuel:	Products:	Temperature (K):	Minimum Mass:	Burning Period:
H	He	4×10^6	0.1	7×10^6 years
He	C, O	1.2×10^8	0.4	5×10^5 years
C	Ne, Na, Mg, O	6×10^8	4	600 years
Ne	O, Mg	1.2×10^9	~8	1 year
O	Si, S, P	1.5×10^9	~8	~0.5 years
Si	Ni - Fe	2.7×10^9	~8	~1 day

On the Earth the most feasible candidate:

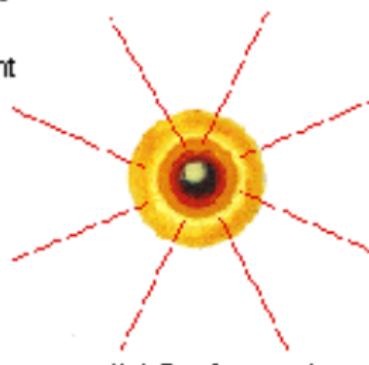
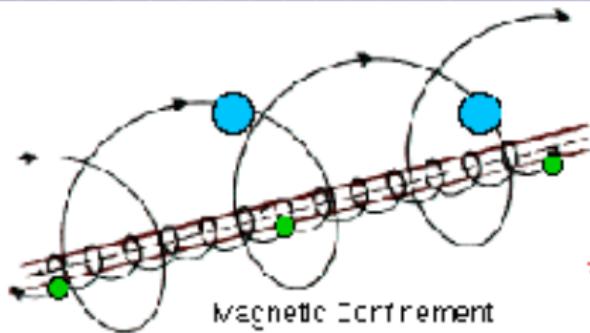


Fundamental forces inventory

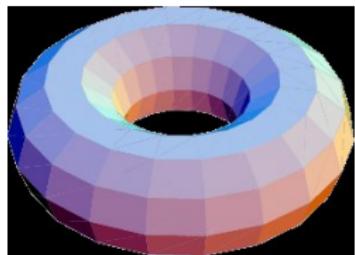
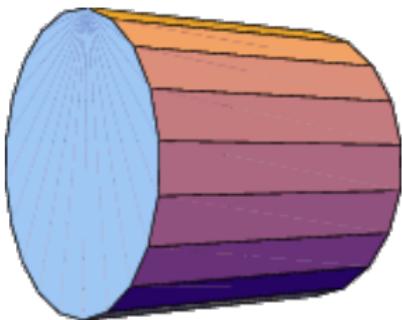
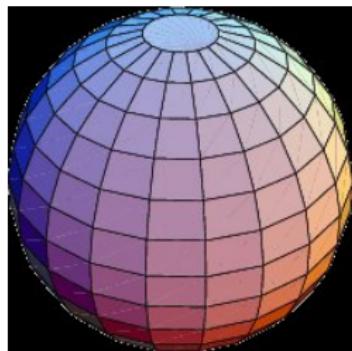
Illustration: Typoform



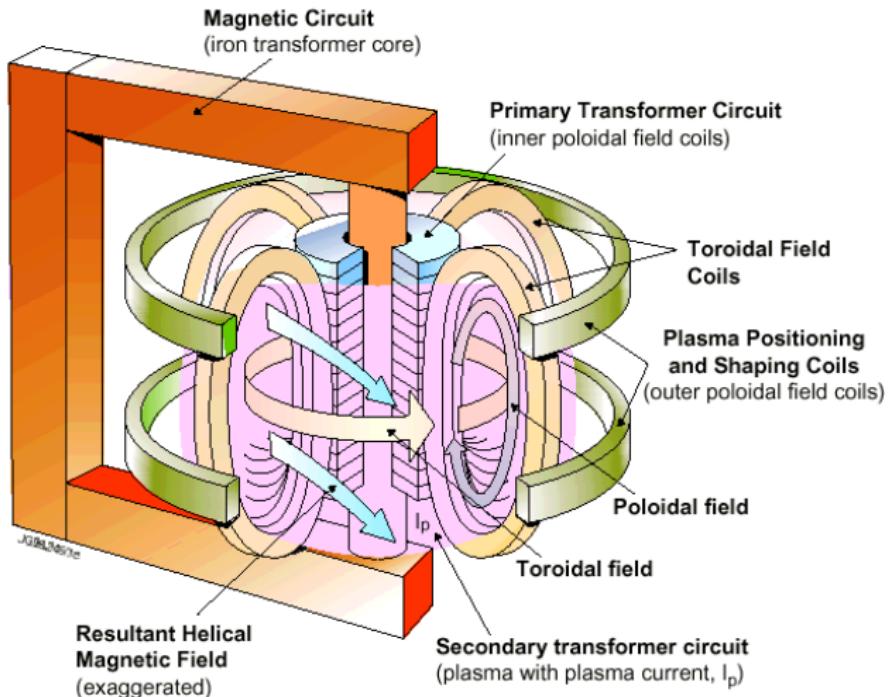
Three ways to confine plasma



Confinement geometry?



Magnetic confinement - Tokamak



Content

1 Introduction

2 Tokamak GOLEM

3 Other Tokamaks

Tokamak GOLEM for Education - Historical Background

Kurchatov Institute near Moscow,
Soviet Union
1960: **TM1-MH**



1974

Institute of Plasma Physics
Czech republic
CASTOR

Culham Centre for Fusion Energy
Great Britain
1989: **COMPASS-D**



2006



2006: new curricula at FNSPE:
**Physics and Technology
of Thermonuclear Fusion**

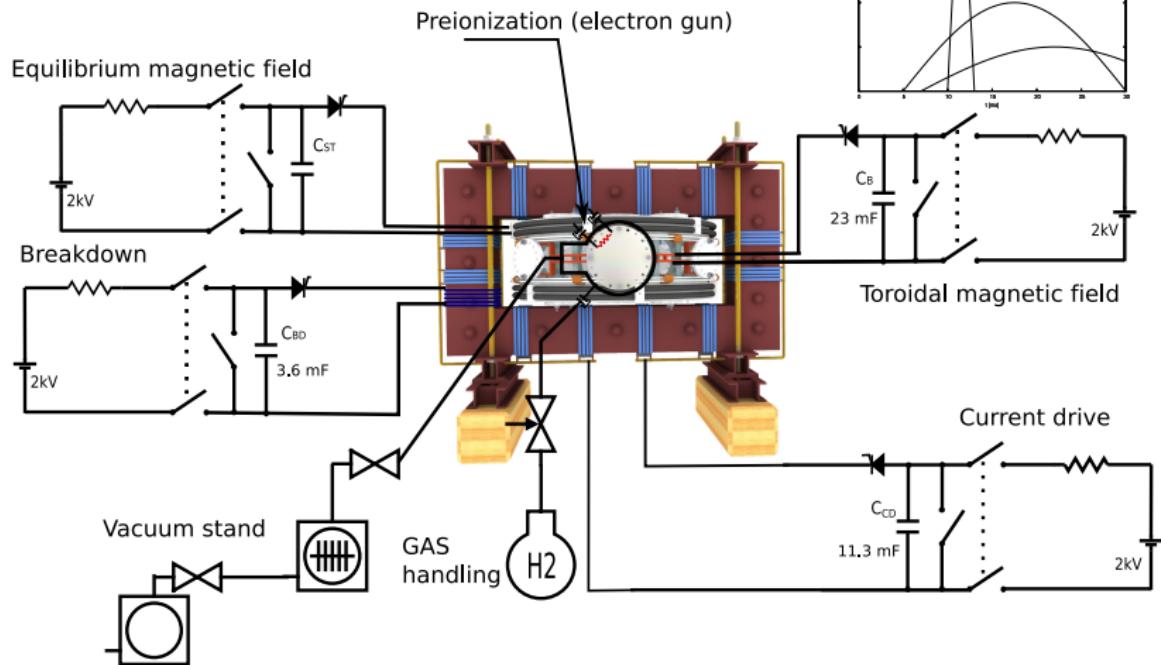
?virtual or real experiments?

2008

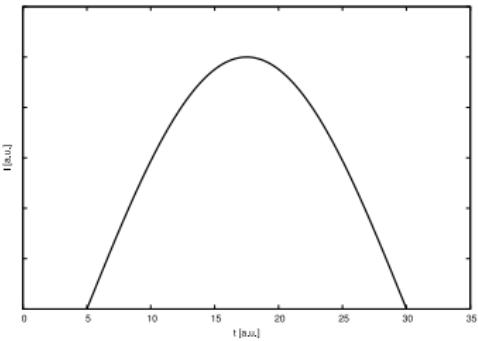
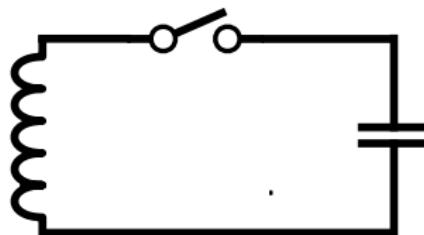
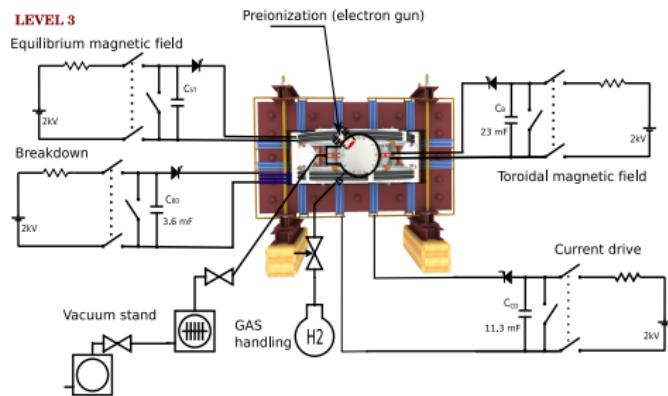
Czech Technical University Prague
Czech republic
GOLEM

Tokamak GOLEM - engineering scheeme

LEVEL 3

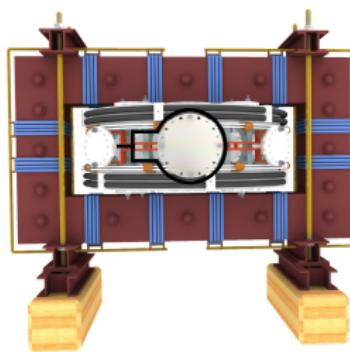


Insertion - LC circuit



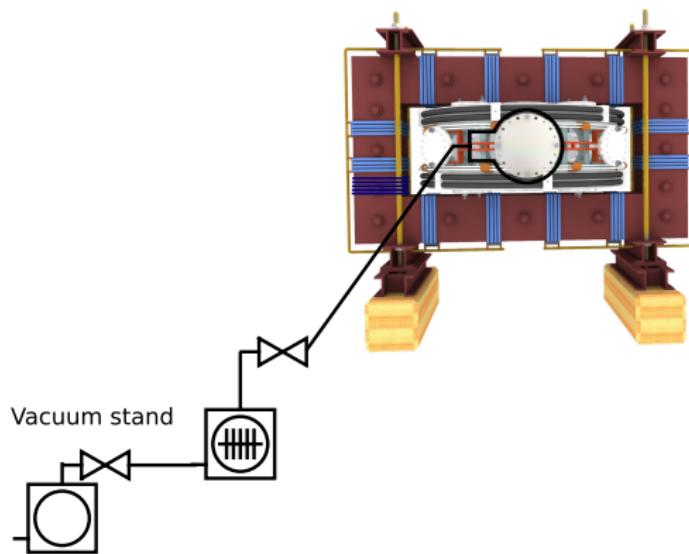
Tokamak GOLEM - basic

LEVEL 0



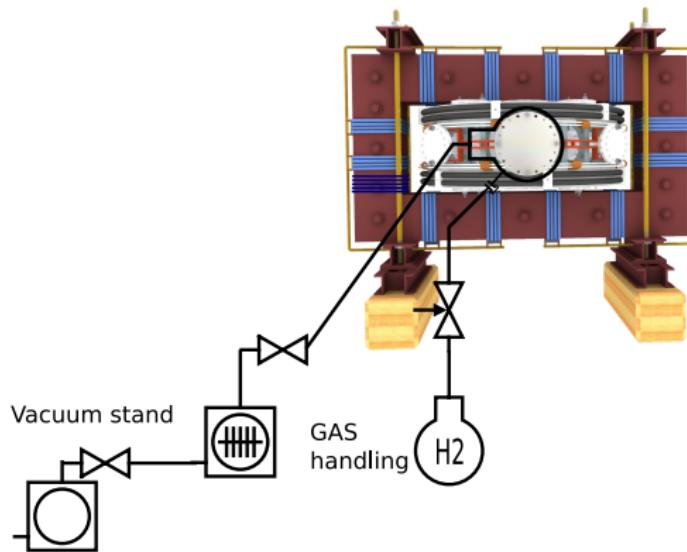
Tokamak GOLEM + vacuum pumping system (100 kPa → 1 mPa)

LEVEL 0



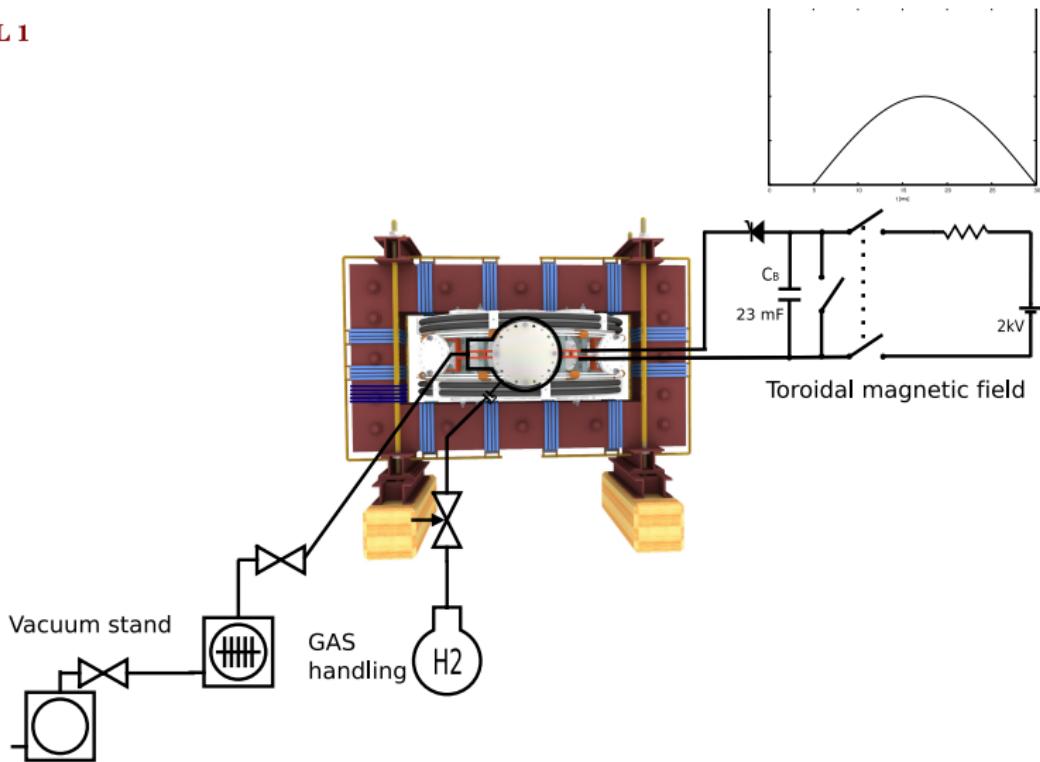
Tokamak GOLEM + working gas management (H₂ or He)

LEVEL 0



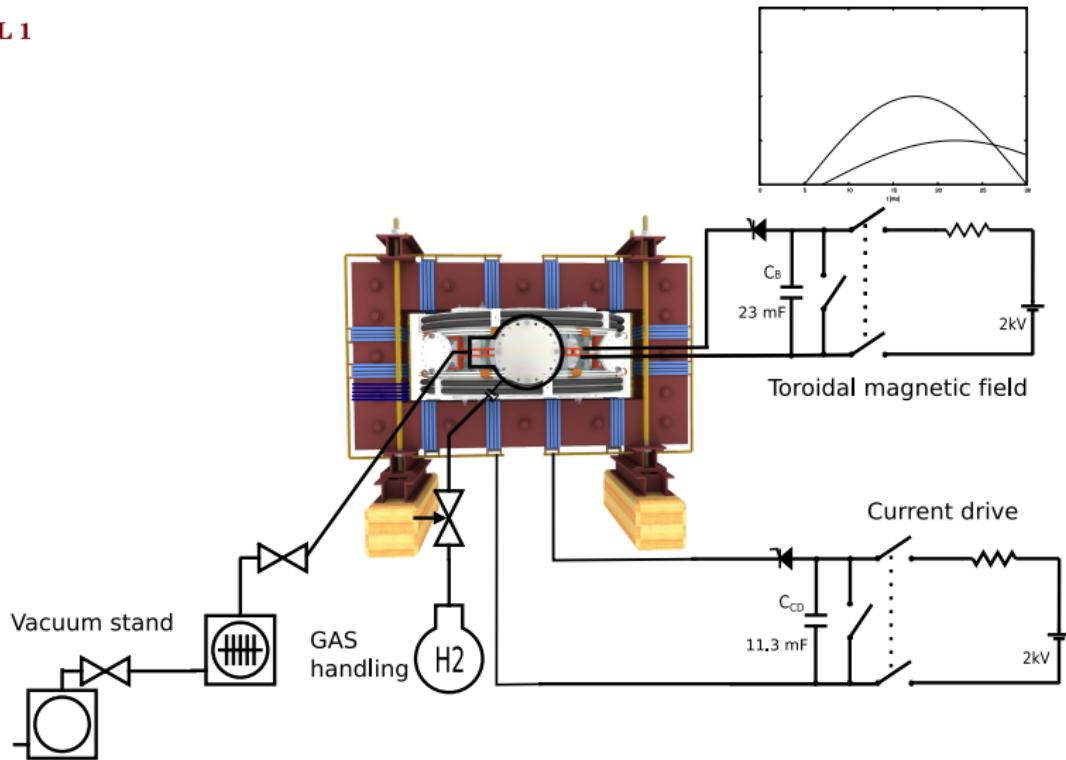
Tokamak GOLEM + B (toroidal magnetic field) Plasma confinement

LEVEL 1



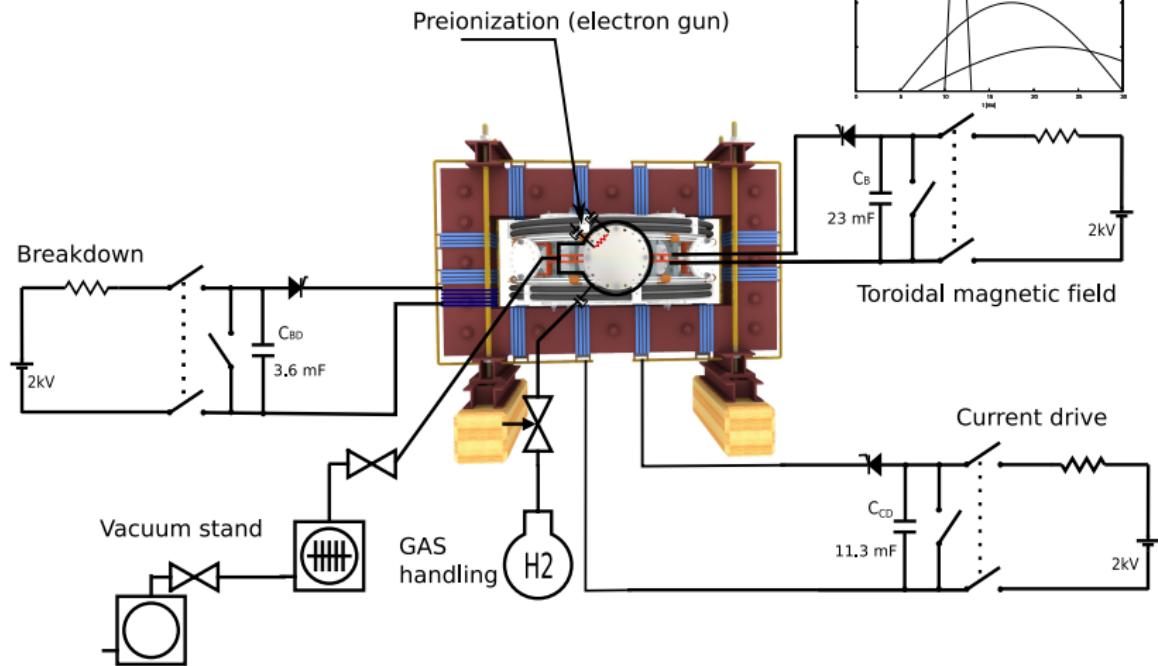
Tokamak GOLEM + CD (toroidal electric field) Plasma heating

LEVEL 1



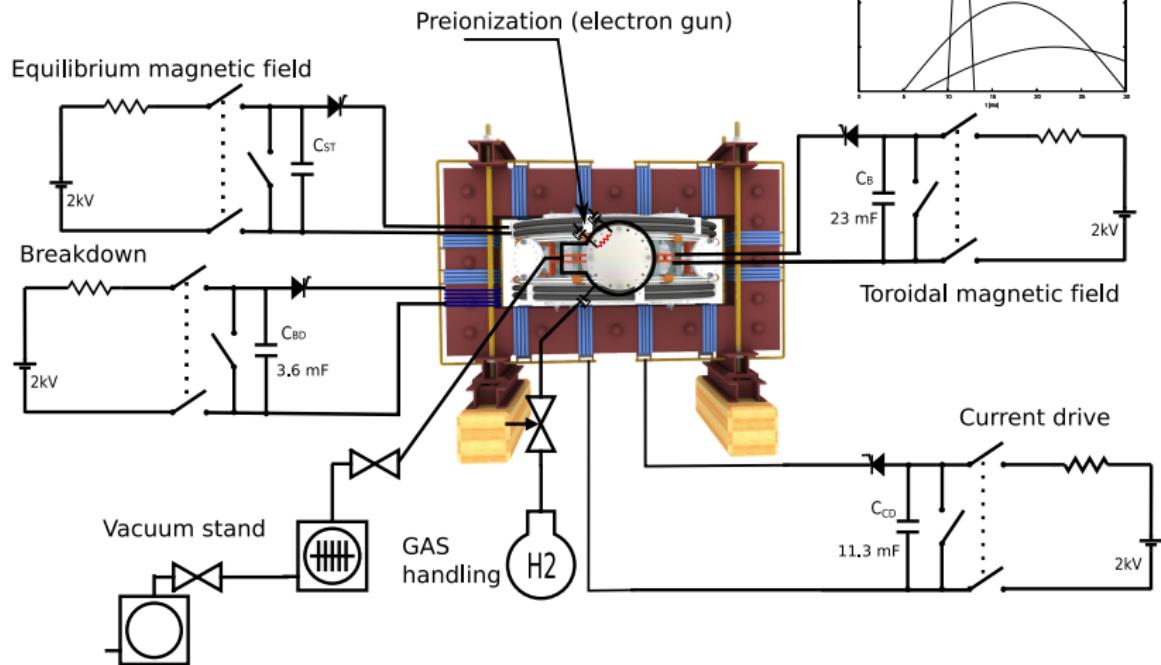
Tokamak GOLEM + breakdown the neutral gas plasma creation

LEVEL 2

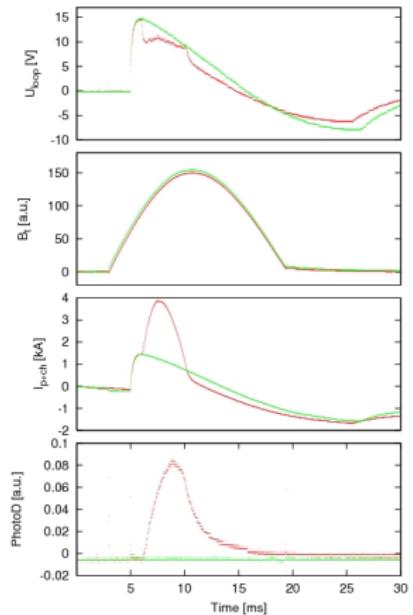
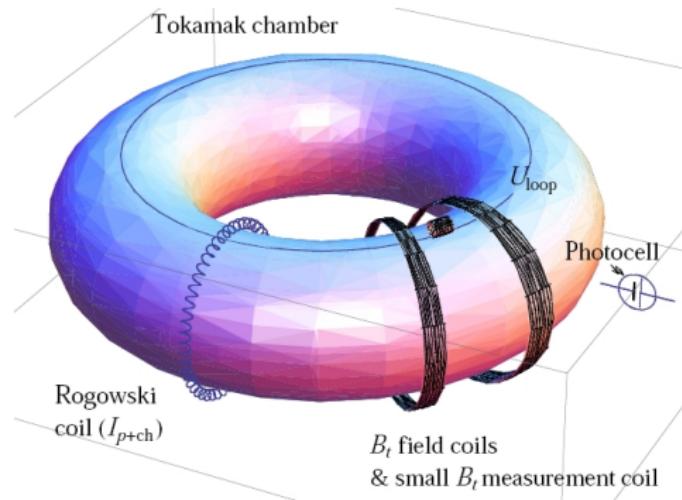


Tokamak GOLEM + plasma stabilization vertical magnetic field

LEVEL 3



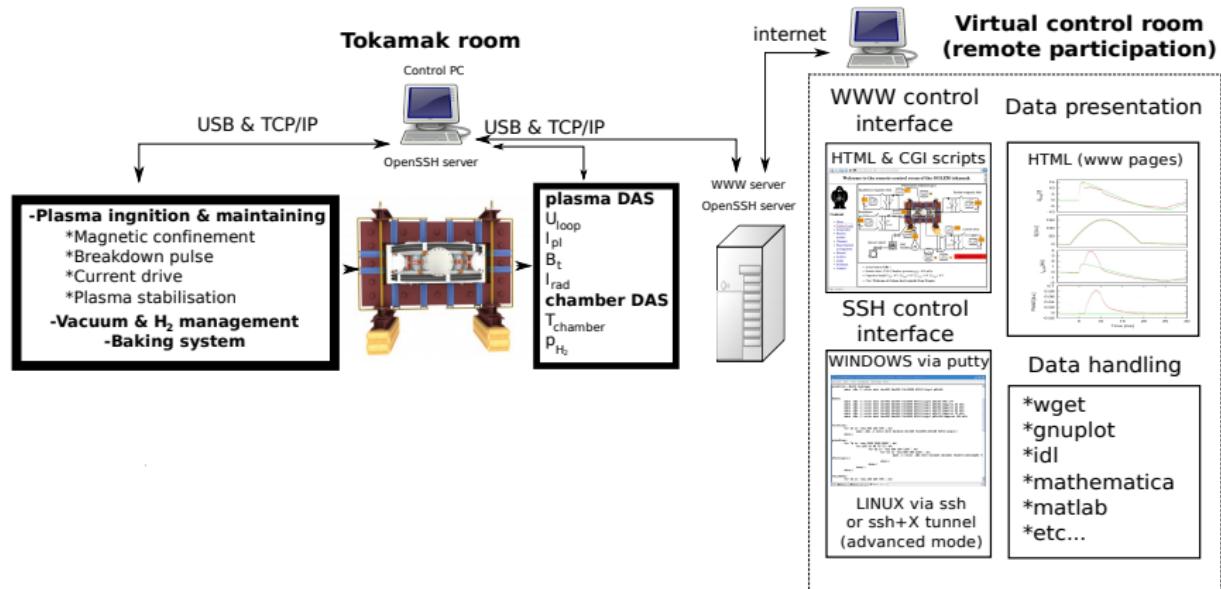
Basic plasma diagnostics in tokamak GOLEM



Data Acquisition System based on:

NATIONAL
INSTRUMENTS

Unique remote operation capability



Central electron temperature estimation I [1]

The current density of plasma is

$$j = E \cdot \sigma \quad (1)$$

where σ is the specific conductivity of plasma given by

$$\sigma(r) = 1.544 \cdot 10^3 \cdot \frac{T_e(r)^{3/2}}{Z_{eff}}, \quad [\Omega^{-1}m^{-1}, \text{eV}] \quad (2)$$

and the electric field E is assumed constant in the poloidal cross-section:

$$E = \frac{U_{loop}}{2\pi R}. \quad (3)$$

Plasma current is obtained by integrating current density over the plasma column:

$$I_{pl} = \int_0^a E \cdot \sigma(r) 2\pi r dr. \quad (4)$$

Central electron temperature estimation II [1]

For the electron temperature, we assume a polynomial profile

$$T_e(r) = T_e(0) \left(1 - \frac{r^2}{a^2}\right)^\alpha \quad (5)$$

where a is the minor radius and $T_e(0)$ is the central electron temperature. Substitution gives us the formula for the central electron temperature

$$T_e(0) = \left(\frac{R}{a^2} \frac{8 \cdot Z_{eff}}{1.544 \cdot 10^3}\right)^{2/3} \cdot \left(\frac{I_{pl}}{U_{loop}}\right)^{2/3}. \quad (6)$$

For the CASTOR/Golem tokamak geometry with $a = 78$ mm :

$$T_e(0) = 89.8 \cdot \left(\frac{I_{pl} [kA]}{U_{loop}}\right)^{2/3} \approx 230 \text{ eV}. \quad (7)$$

The effective ion charge is assumed as $Z_{eff} = 2.5$.

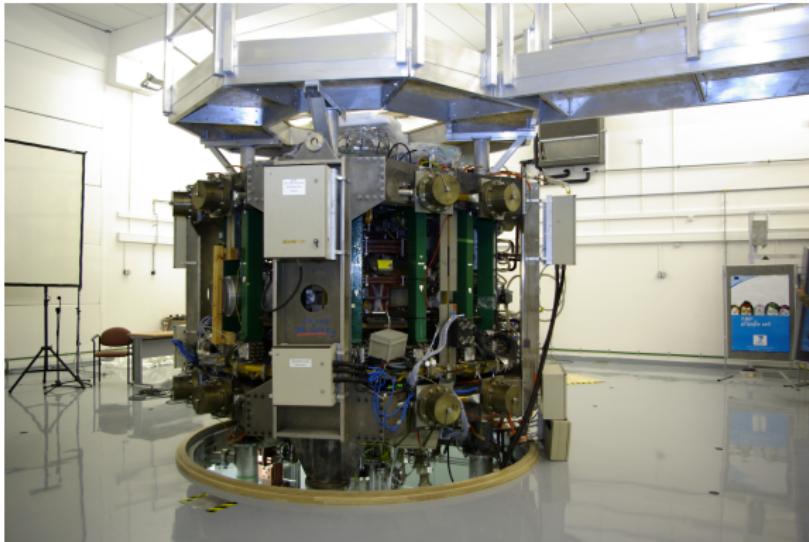
Content

1 Introduction

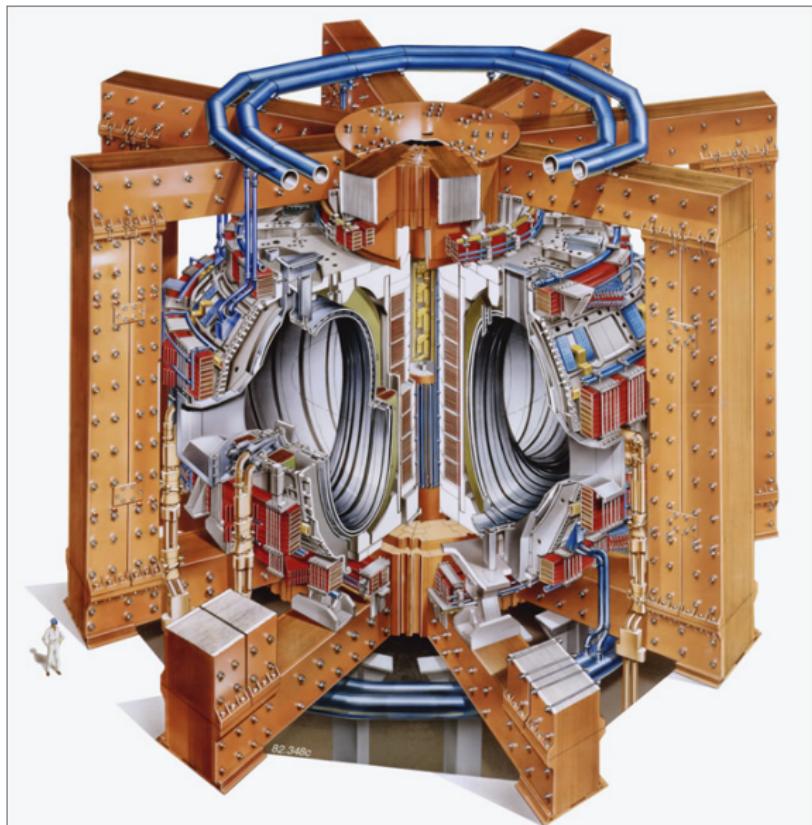
2 Tokamak GOLEM

3 Other Tokamaks

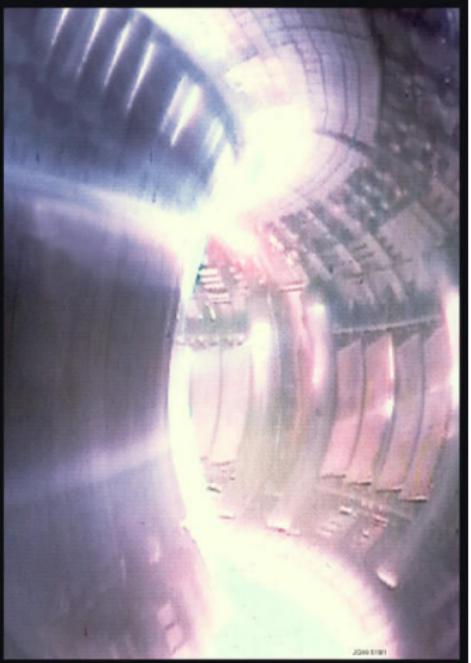
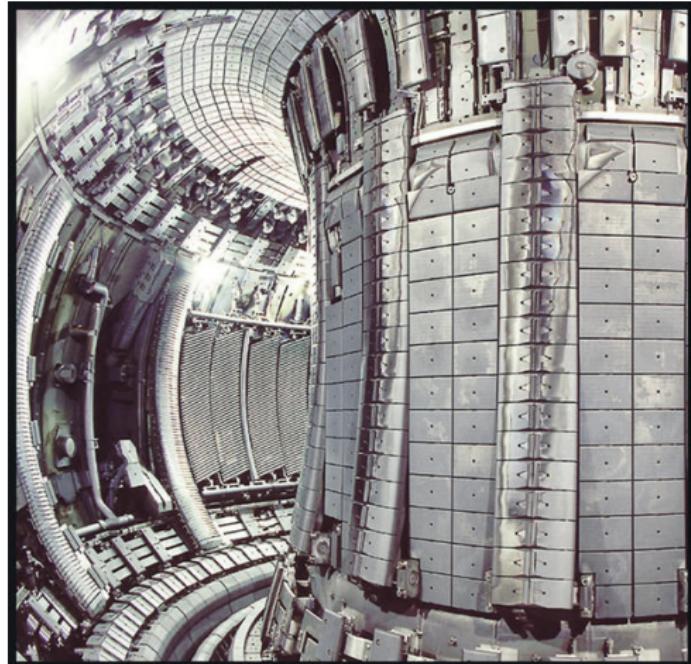
Compass



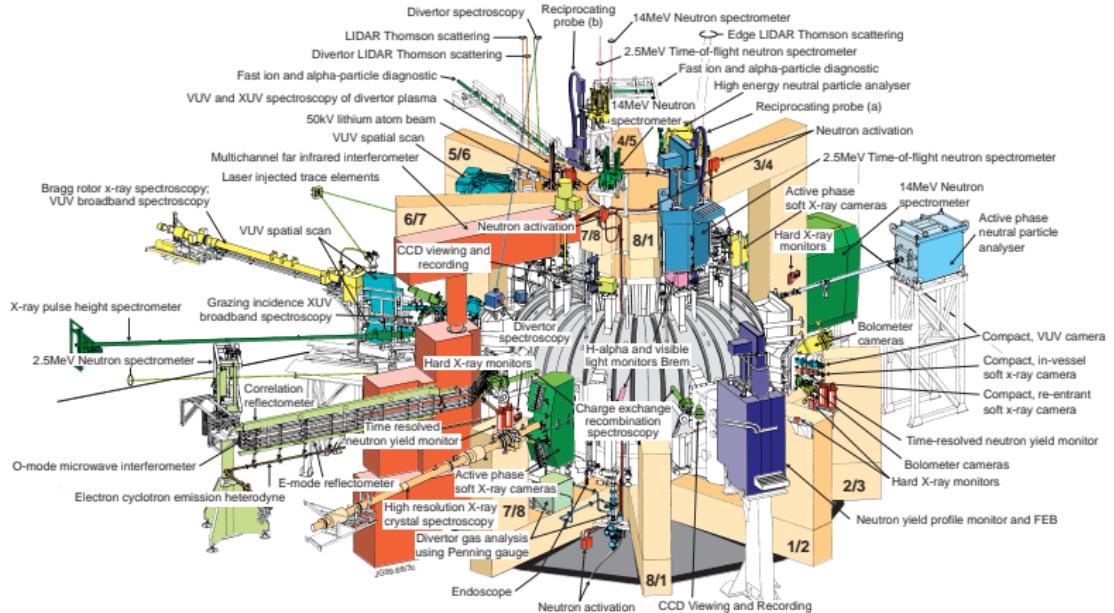
JET



Plasma discharge in tokamak



Joint European Torus (diagnostics)

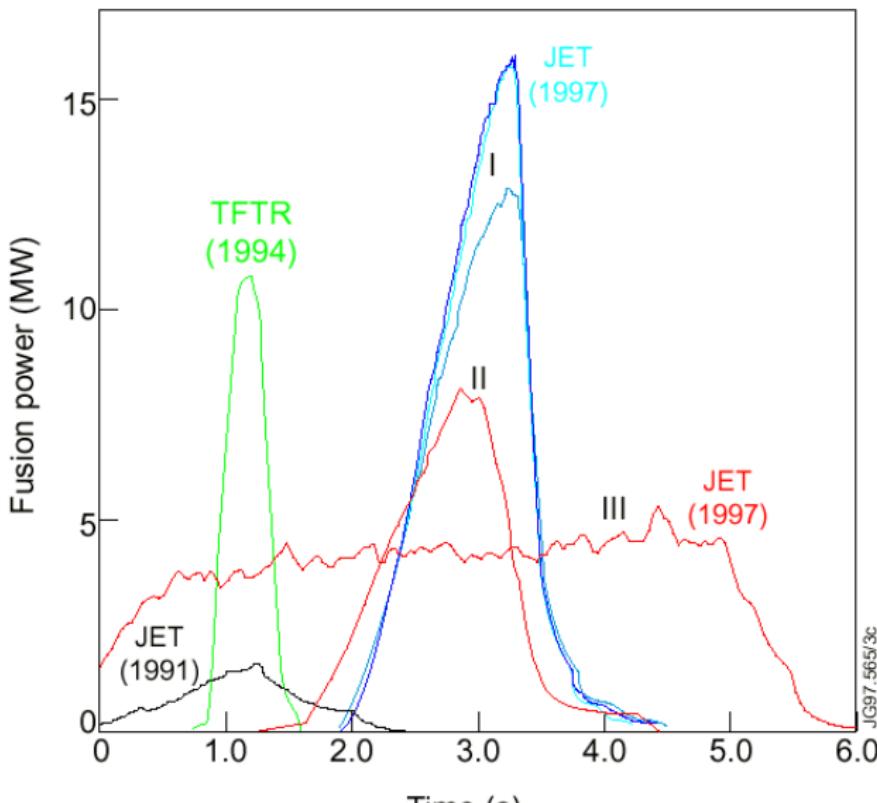


The challenge of characterising extreme conditions of nuclear fusion plasmas both spatially and temporally has inspired JET to produce an impressive array of diagnostic techniques. Drawing from fields as diverse as neutronics, spectroscopy, lasers and microwaves, JET is a leader in the art of measurement."

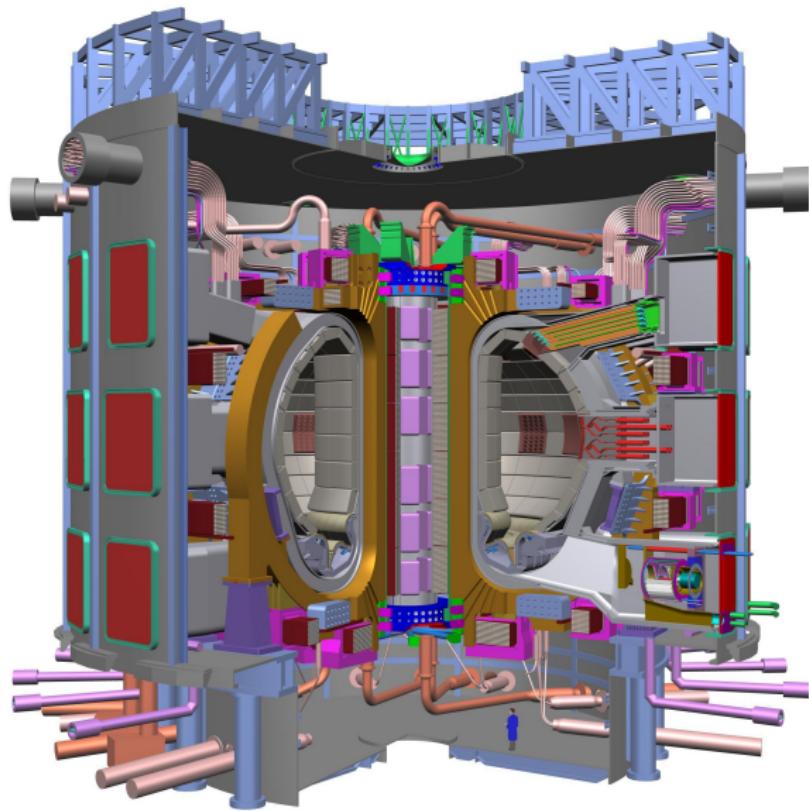


Andrea Murari, Task Force Leader - Diagnostics.

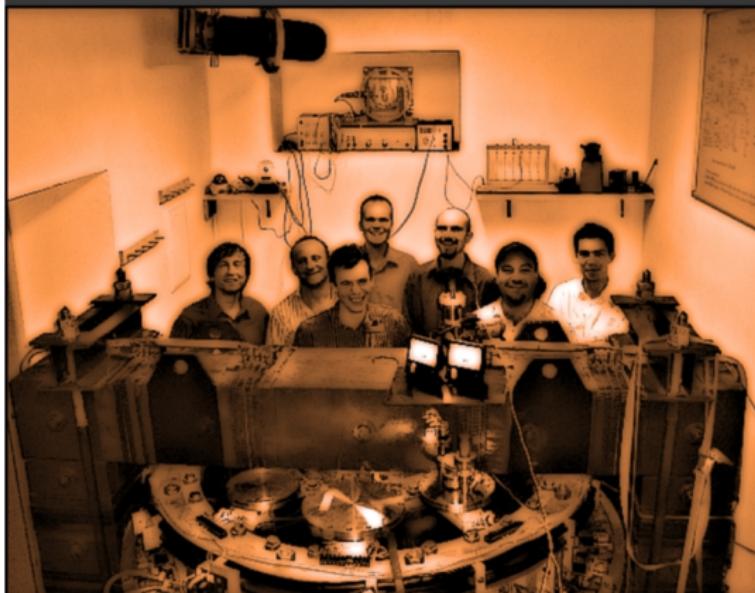
Fusion power at the JET (DT fuel)



ITER



14.9.2009; ČVUT; PRAHA ...



Acknowledgement

Acknowledgement

The financial support by FUSENET, MSM 6840770039, MSM 6840770014 and A1581 is acknowledged.

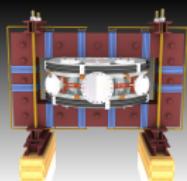
Special thanks to the GOLEM team (students, teachers, technicians)

Edita Bromova, Zdenek Cespiro, Ivan Duran, Vladimir Fuchs,
Ondrej Grover, Pavel Hacek, Billy Huang, Igor Jex, Michal Kazda,
Jindrich Kocman, Martin Kubic, Ondrej Kudlacek, Petr Liska,
Tomas Markovic, Jan Mlynar, Michal Odstrcil, Tomas Odstrcil,
Ondrej Pluhar, Gergo Pokol, Ondrej Sebek, Adam Sindlery, Michal
Smid, Gabriel Vondrasek, Frantisek Zacek, and Jiri Zara.

Winter school of Plasma Physics - Marianska 2011 (Tokamak, probably COMPASS, with NBI)



Thank you for your attention



<http://golem.fjfi.cvut.cz>,
you and your students are welcome

References I

-  Brotankova, J.
Study of high temperature plasma in tokamak-like
experimental devices.
PhD. thesis 2009.
-  Tokamak GOLEM at the Czech Technical University in
Prague.
<http://golem.fjfi.cvut.cz>, 2007.
-  V. Svoboda, B. Huang, J. Mlynar, G.I. Pokol, J. Stockel, and
G Vondrasek.
Multi-mode Remote Participation on the GOLEM Tokamak.
Fusion Engineering and Design, 86(6-8):1310–1314, 2011.